

Edge Plasma Heat Flux Study in the COMPASS tokamak

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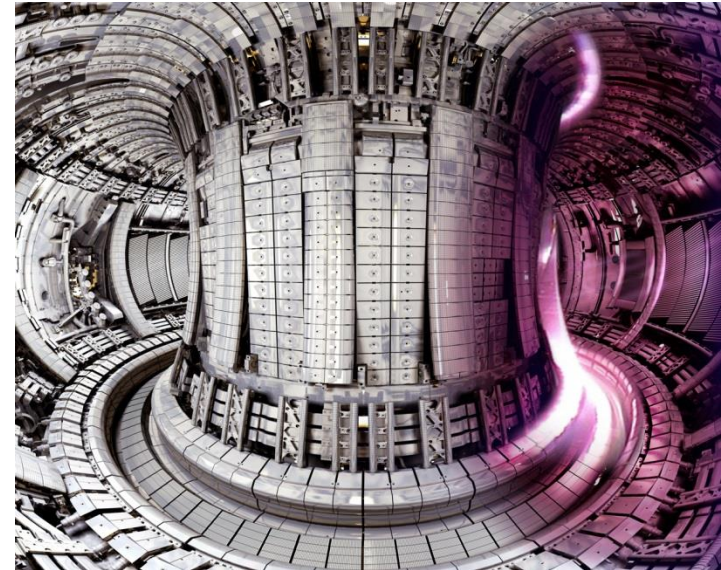
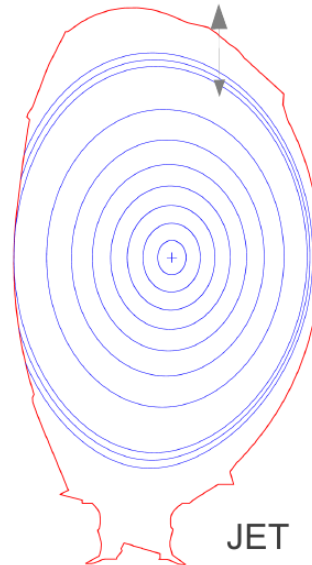
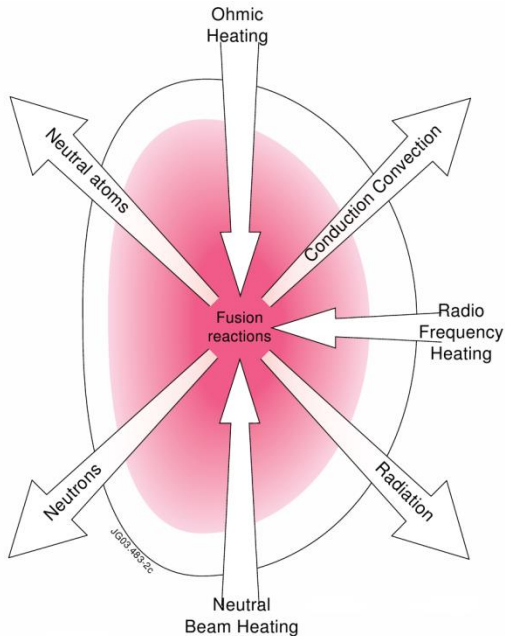
Outline:

- **Parallel Heat Flux** in SOL
- Motivation – **ITER start-up limiter shape** optimization
- Special HFS limiter experiments at COMPASS
 - **Funnel effect** study
 - **double-exponential SOL profile** investigation
 - limiter shape optimization
 - **Non-ambipolar flux** study
 - **Heuristic Drift (HD) model**

- Diffusion, convection and turbulent transport brings particles to the SOL
- ⇒ **parallel heat flux** with a **radial decay** λ_q occurs in the SOL

$$q_{\parallel}(r) = q_{\parallel 0} \exp(-r/\lambda_q) \quad q_{\parallel 0} \approx \frac{P_{\text{SOL}} (B/B_p)}{4\pi R \lambda_q}$$

- Radial profile of the parallel heat flux determines **limiter heat load** distribution ⇒ $q_{\perp \text{lim}} = q_{\parallel} \sin(\alpha)$



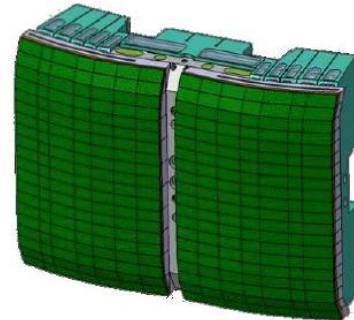
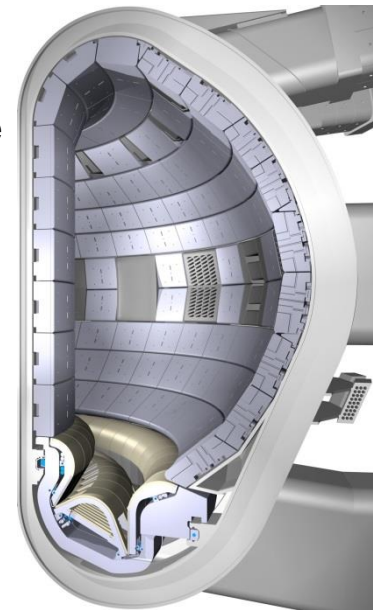
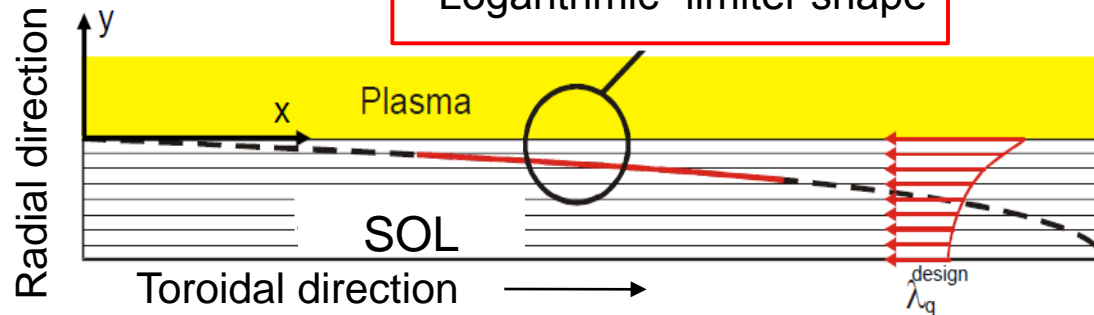
ITER start-up limiter shape optimization

- HFS beryllium limiter tiles are foreseen to be used for plasma start-up in ITER
- **Tile shaping** should provide **uniform heat flux** distribution across the limiter surface
- **Exponential radial decay of parallel heat flux** in SOL is expected

$$q_{\perp \text{lim}} = q_{\parallel 0} \exp(-r/\lambda_q) \sin(\alpha)$$

$$y = -\lambda_q \ln(1 - Cx/\lambda_q)$$

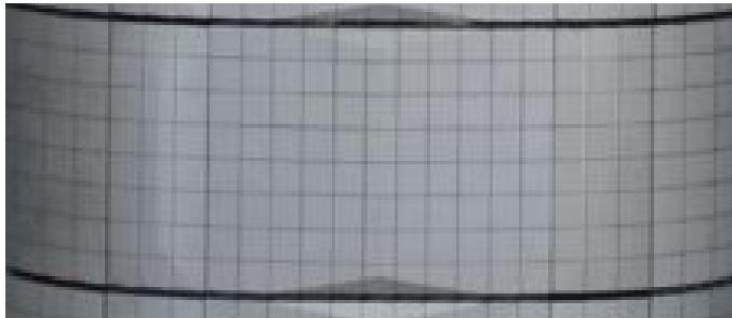
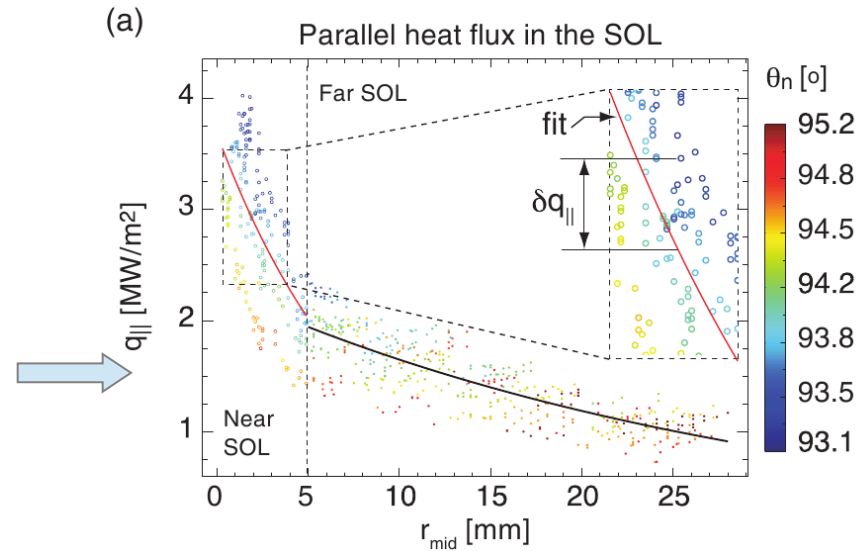
“Logarithmic” limiter shape



Prediction of λ_q for ITER

- 1) Current limiter design $\lambda_q^{\text{imp}}=50\text{mm}$
- 2) Multi-tokamak scaling $\lambda_q^{\text{omp}}\sim 45\text{mm}$
(7.5MA scenario) [Horacek et al. 2013]
- 3) JET beryllium ITER-like limiter experiments
 $\lambda_q\sim\text{couple mm}$
Near & far SOL feature observed [Arnoux 2012]

**Unexpected heat flux observed
- limiter melted at the crown!!!**



Funnel effect??? Non-single exponential SOL profile??? Non-ambipolar flux???

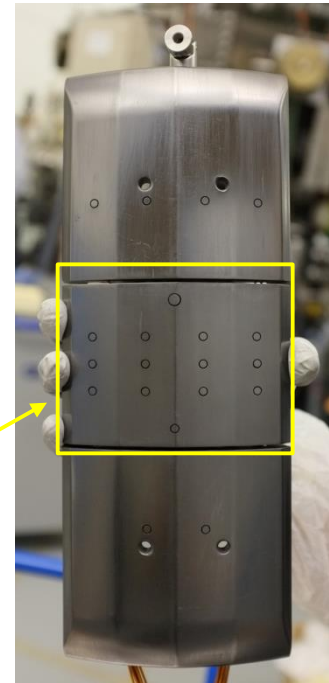
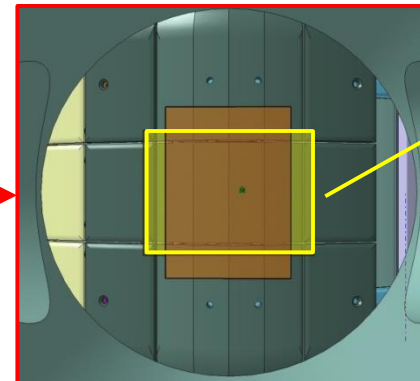
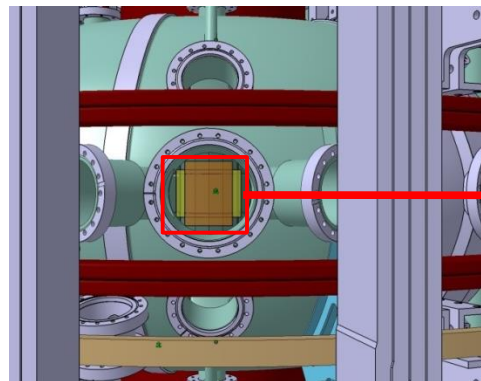
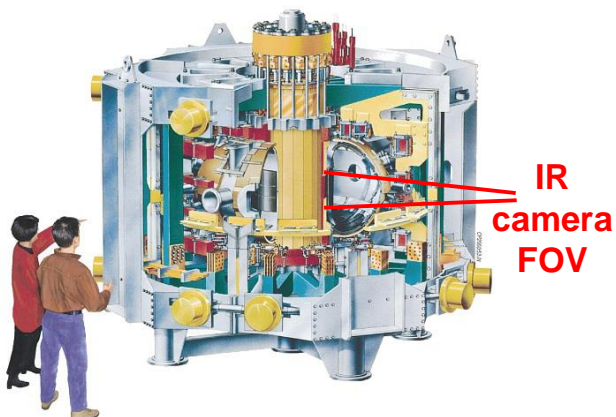
Series of dedicated experiments with **special limiters** at HFS

Main diagnostics:

- Microbolometric **IR kamera** (120Hz, 160x120px, 1px~1mm)
+ **THEODOR code** for heat flux calculation
- Midplane **reciprocating probe manipulator**
- 20 embedded **HFS limiter probes**

Various I_p and B_T orientations

Radial position of the limiter was varied ($\Delta R = 12, 22, 40\text{mm}$)



Assumption:

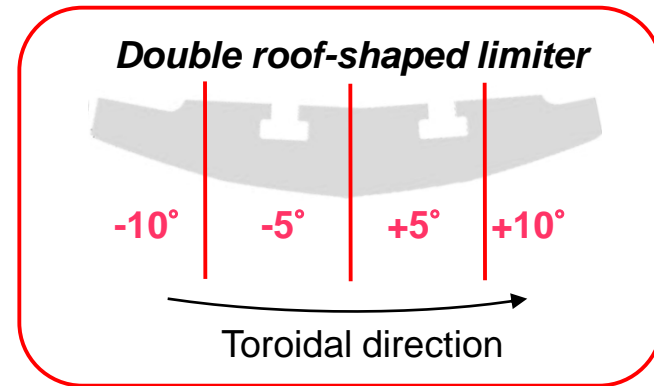
„Funnel effect“ is angle-dependent

=> additional heat flux to the 5° parts expected

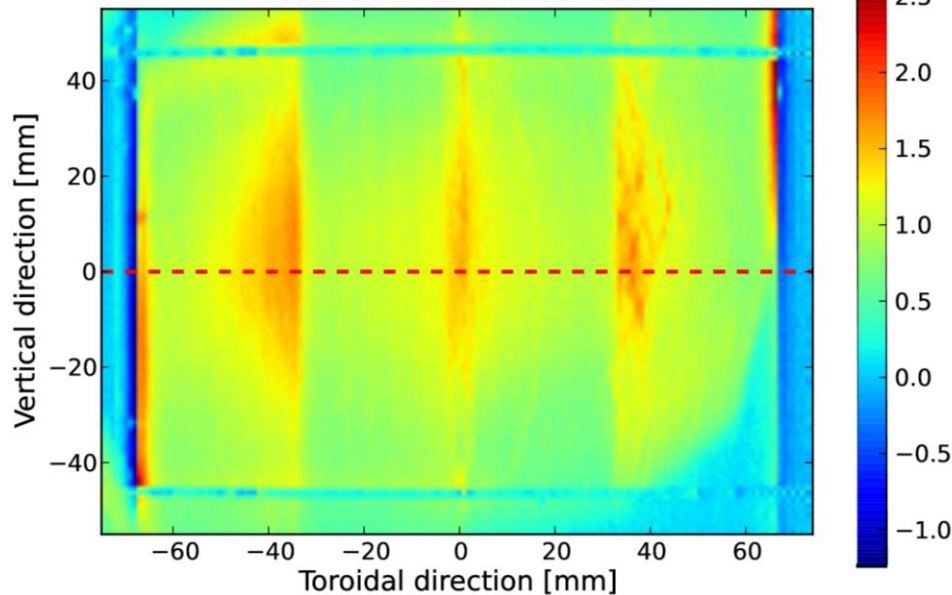
Result:

Heat flux purely parallel

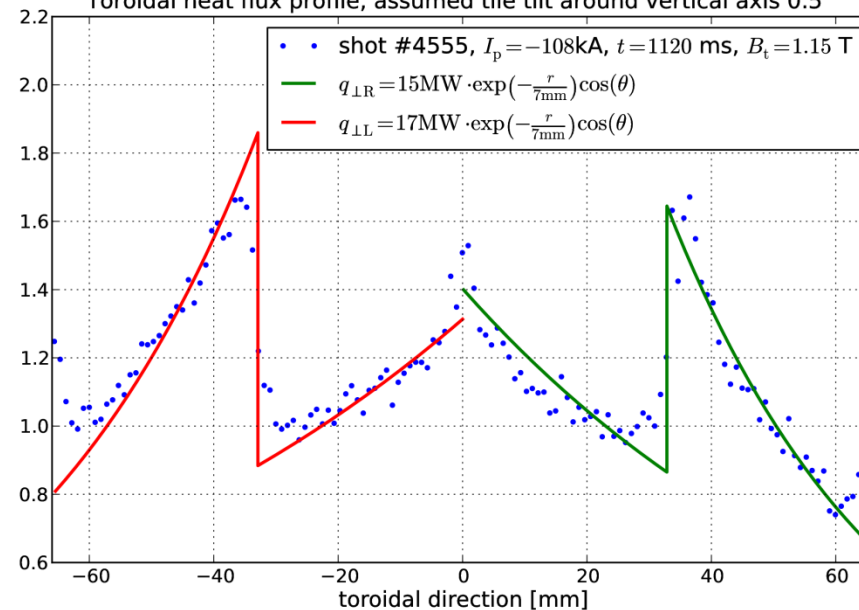
=> no funnel effect evidence



Heat flux from Theodor code



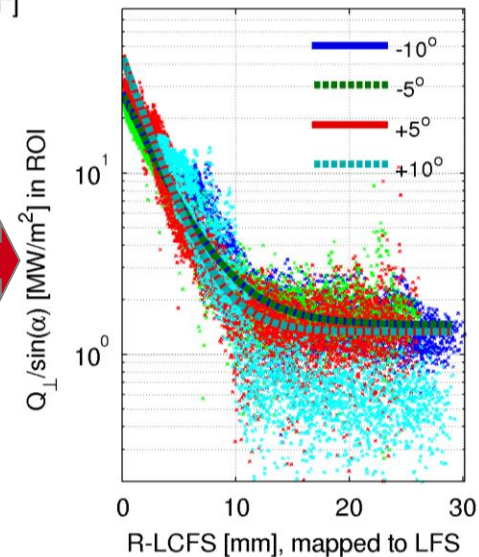
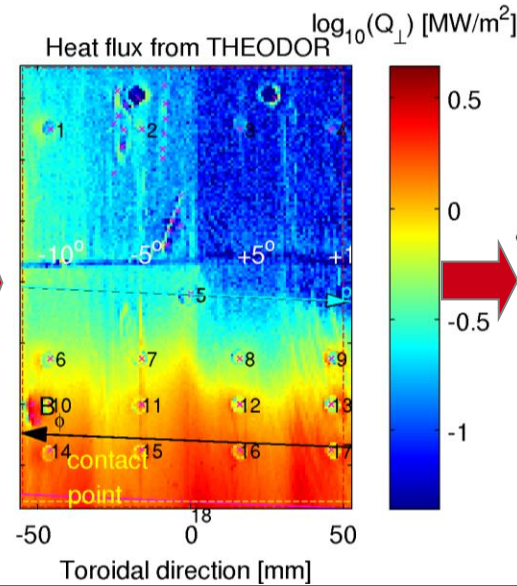
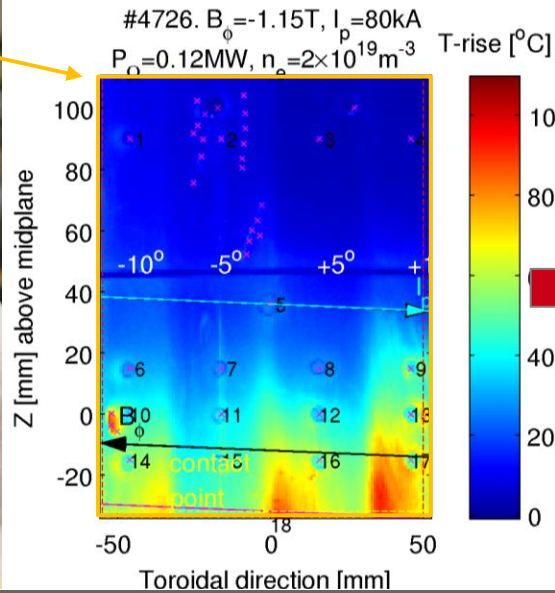
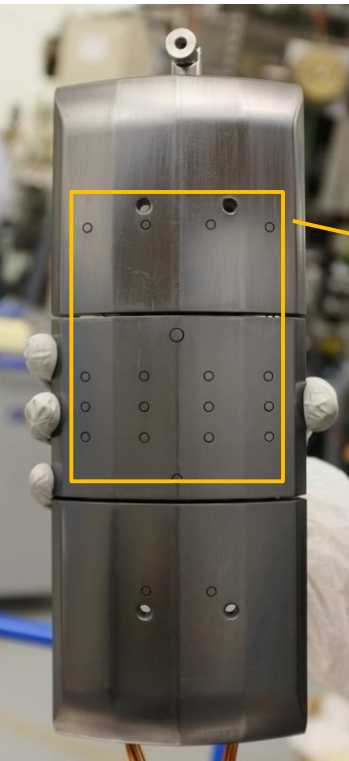
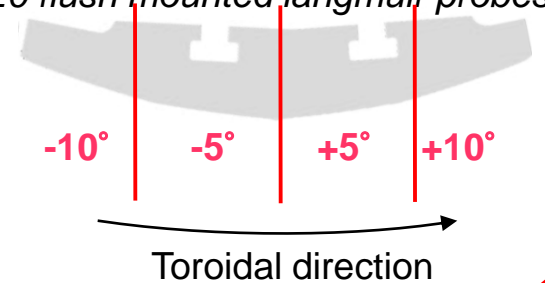
Toroidal heat flux profile, assumed tile tilt around vertical axis 0.5°

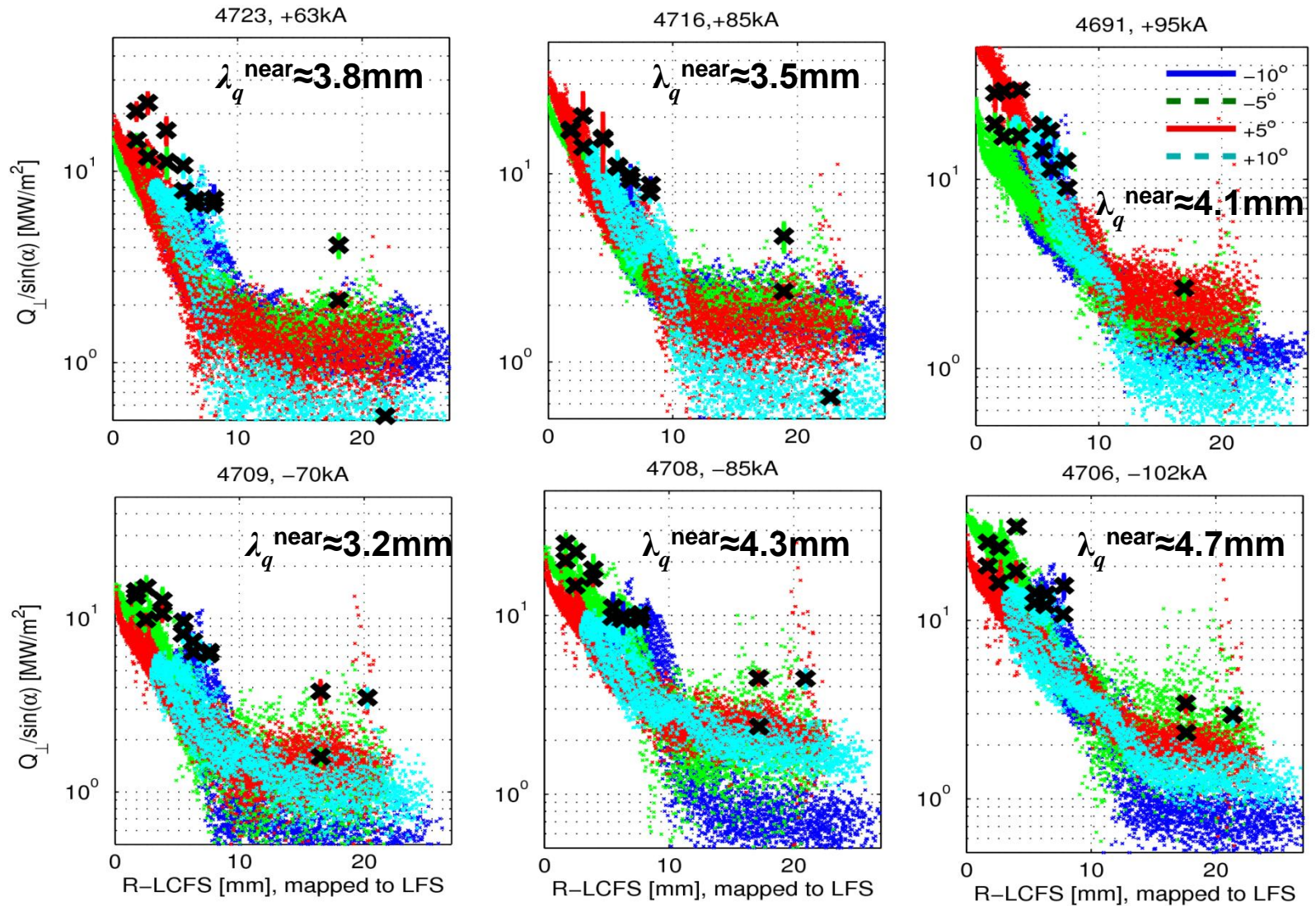


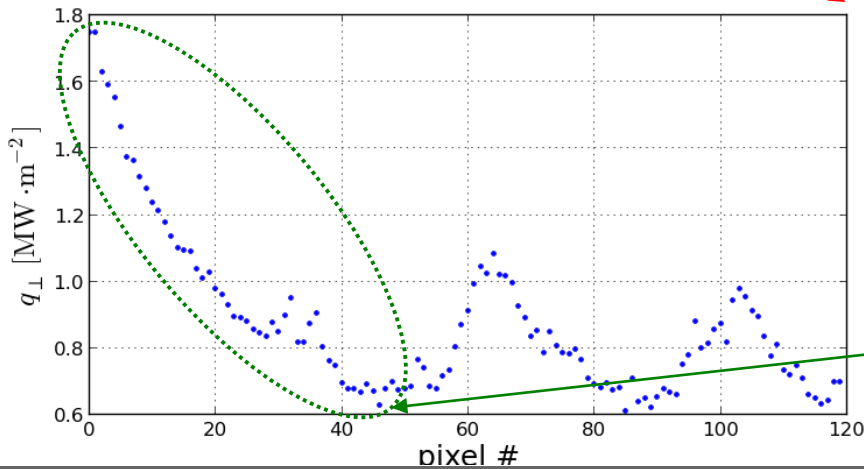
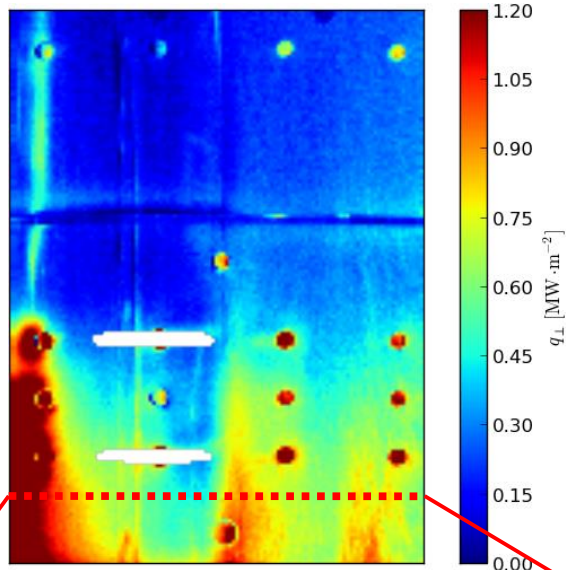
Double-exponential radial profile:

$$q_{\parallel}(\Delta r) = q_{\parallel 0, \text{near}} \exp(-\Delta r / \lambda_{q, \text{near}}) + q_{\parallel 0, \text{far}} \exp(-\Delta r / \lambda_{q, \text{far}})$$

Double roof-shaped limiter
 $\pm 5^\circ$ tilting mechanism
 20 flush mounted langmuir probes





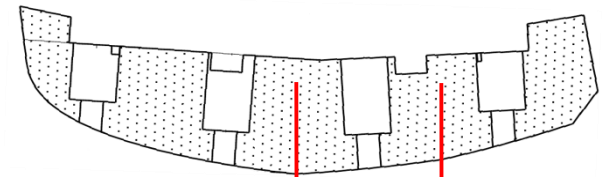


Combined double roof-shaped and logarithmic limiter

$\pm 5^\circ$ tilting mechanism

4 flush-mounted langmuir probes

16 domed langmuir probes



Log part

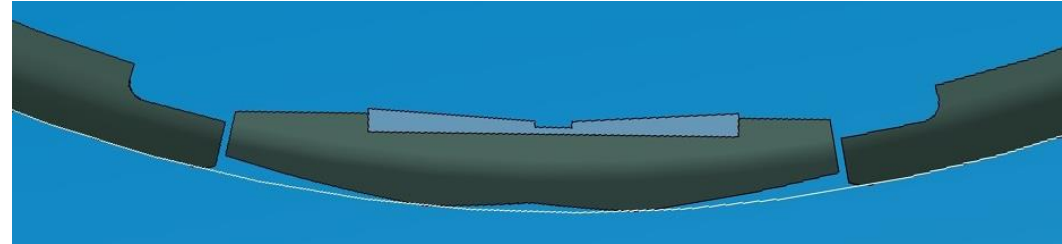
$+5^\circ$

$+10^\circ$

Toroidal direction

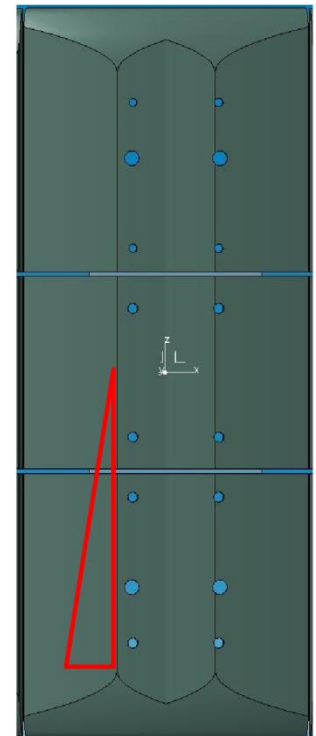
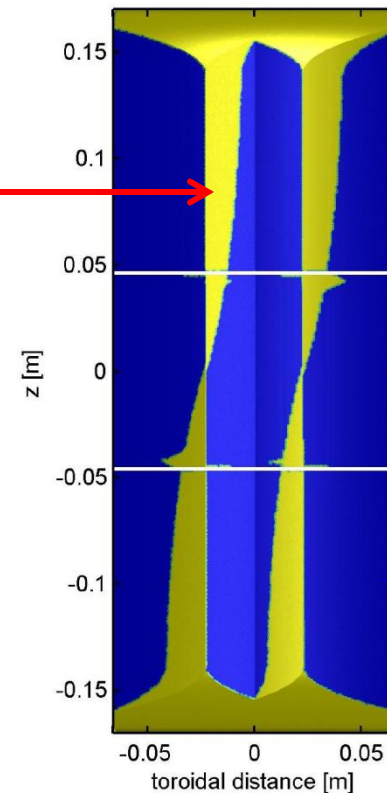
Log part of the limiter optimized for $\lambda_q^{\text{omp}}=2.9\text{mm}$
=> heat flux should increase towards limiter edge for typical COMPASS shot

Plan for 01/2014

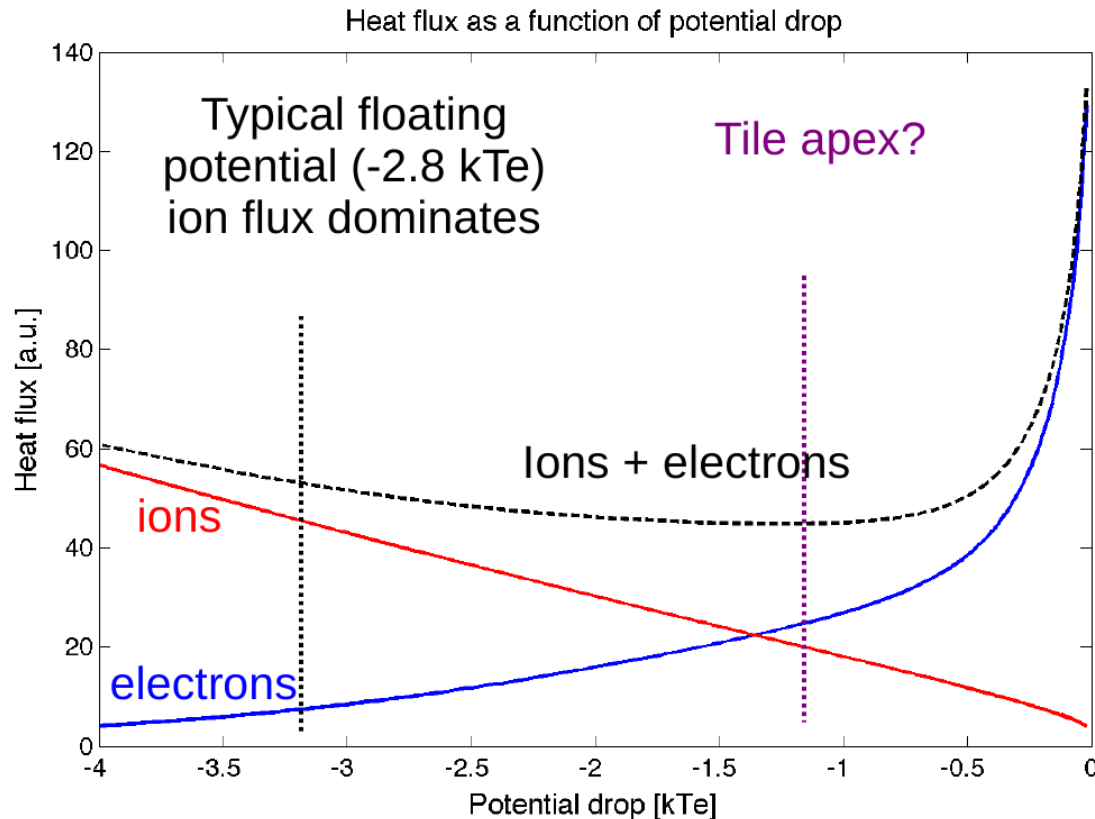


- Better approximation of the ITER continuous central column limiter
 - Quite **small wetted area** →
- ⇒ IR camera with a finer spatial resolution will be used

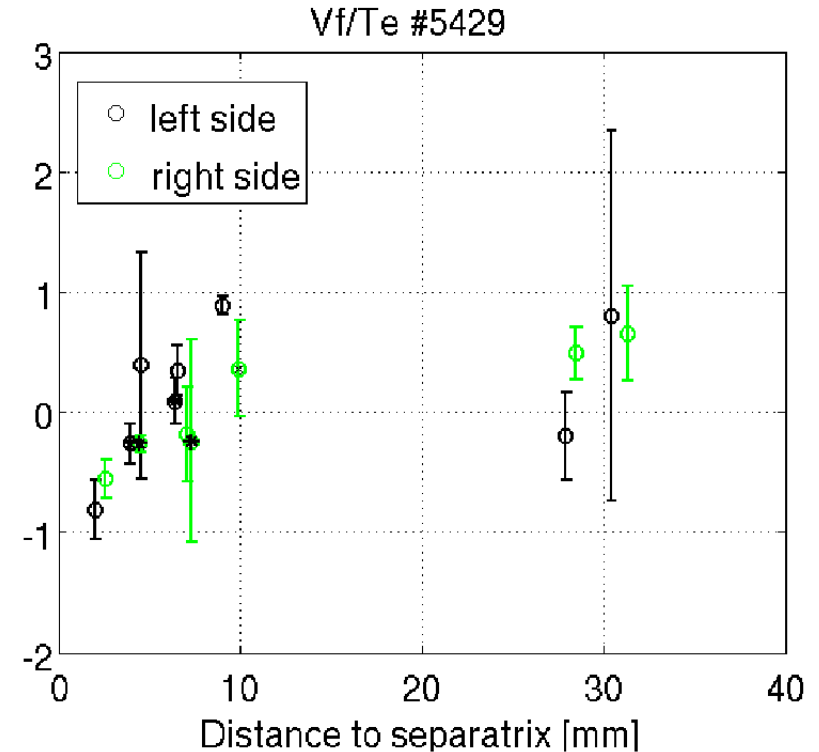
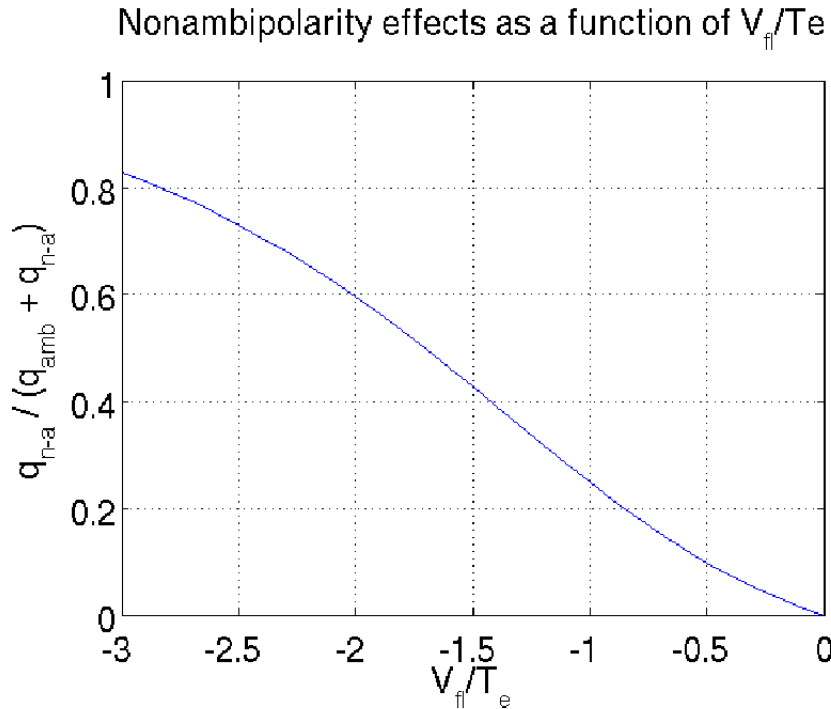
JADE MWIR camera
cooled InSb detector
320x256px. 170Hz 1px.~0,6mm
Larger FOV = wider radial profile



- Local heat flux exposed to the limiter depends on its potential compared to the plasma potential
- Heat flux can significantly vary as the plasma potential varies across the limiter surface



Role of floating potential

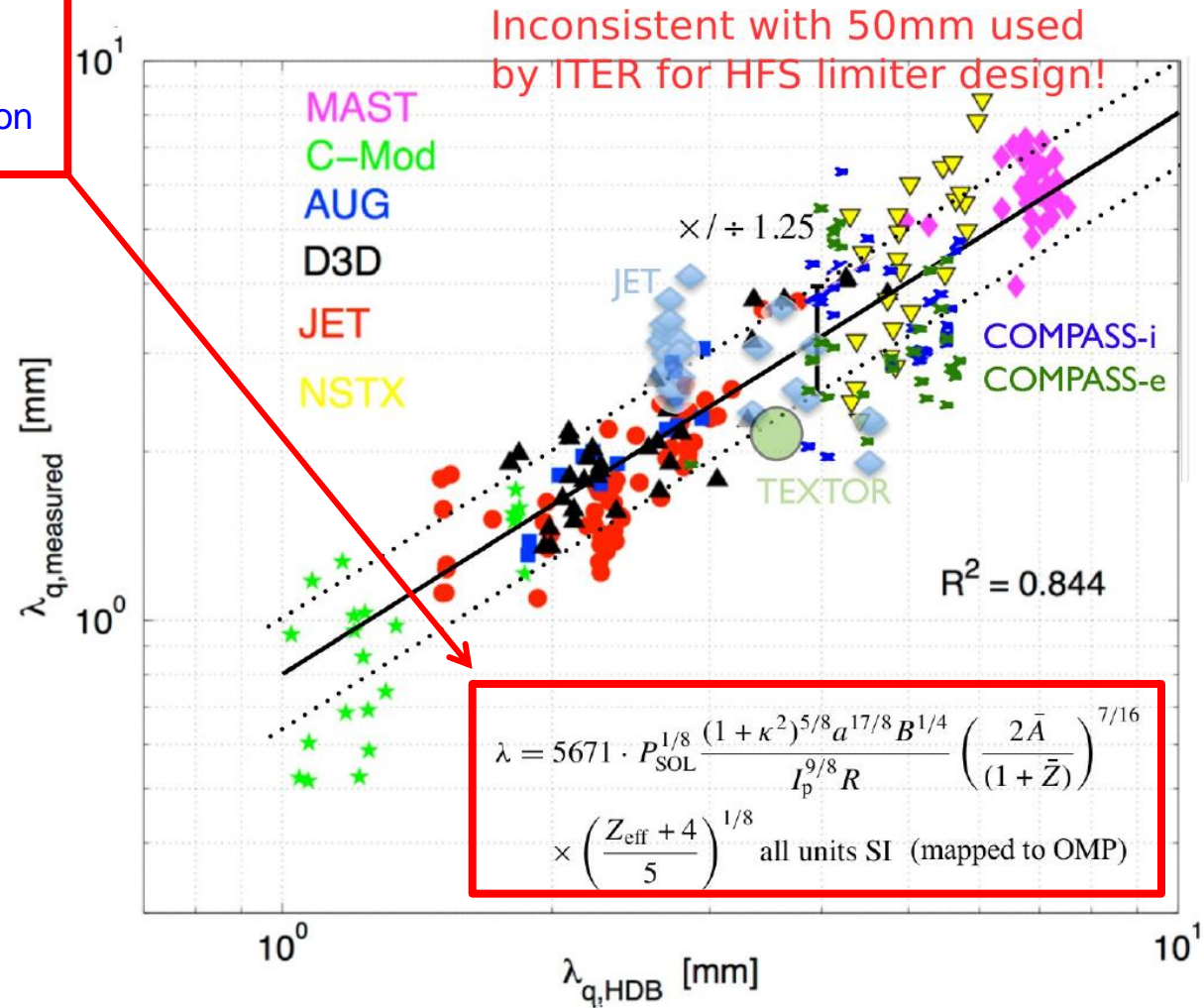
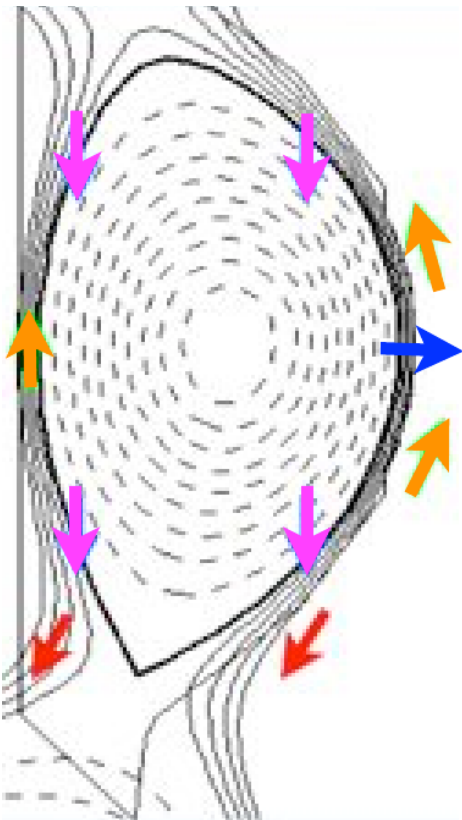


In order to have Δq comparable to q , we need $V_{fl}/T_e < -1.5$

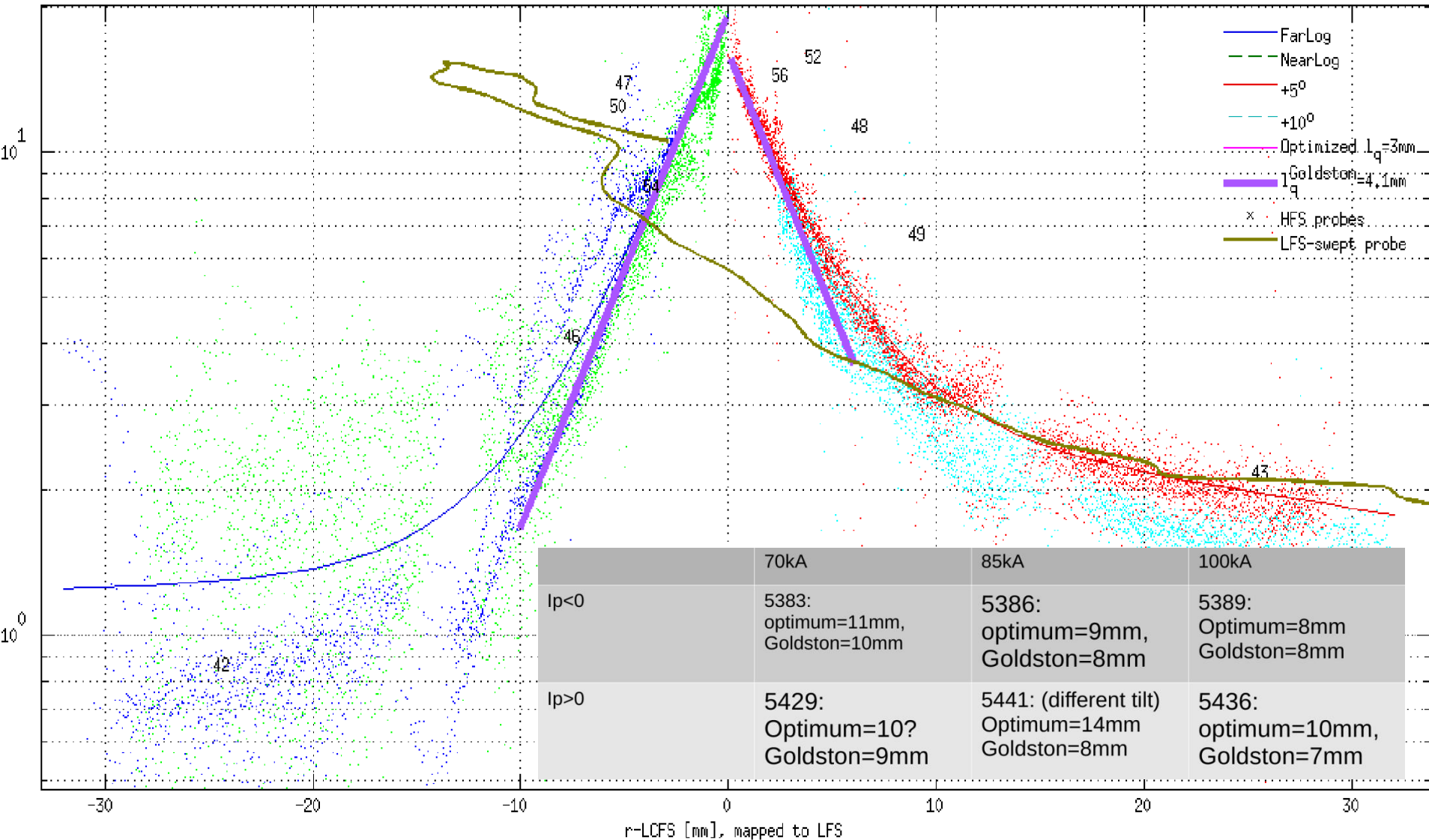
In experiment we generally see $V_{fl}/T_e > -1.0$

But still, non-ambipolarity may contribute ~ up to 30% in the near SOL

- Vertical drifts cross separatrix into SOL
- Parallel flows in SOL
- Pfirsch-Schlüter flow
- Anomalous electron thermal conduction

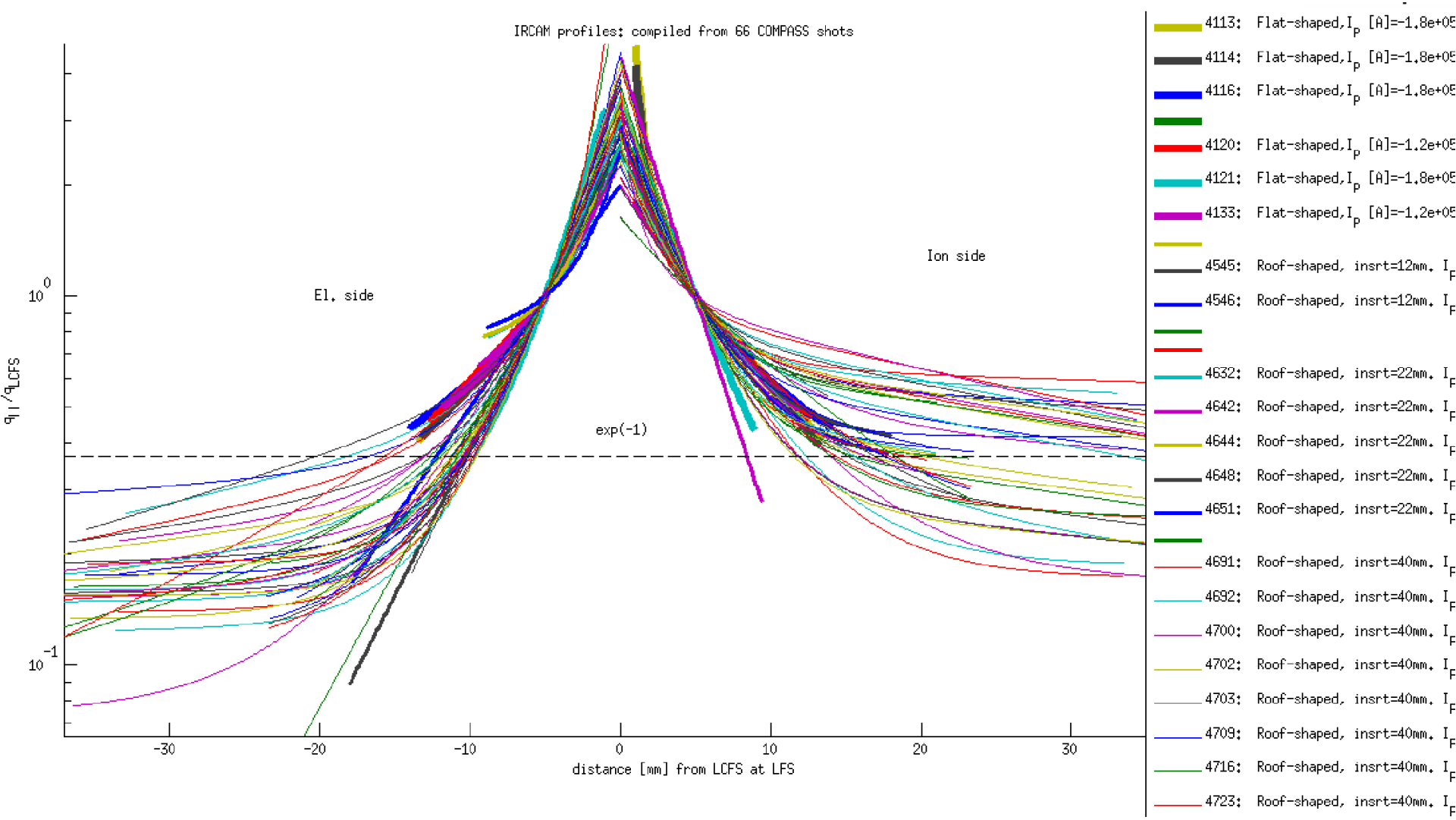


$Q_{\perp}/\sin(\alpha)$ [MW/m²] in ROI, 5389



- COMPASS experiments indicated that ITER HFS limiter shape optimized for decay length measured at LFS might not be correct - we need more data analysis
- Further study of double-exponential heat flux SOL profile of plasma limited at HFS is needed.
- **λ_q scaling based on HFS data (IR or probes) is urgently needed!**
- **Local non-ambipolarity** of the parallel heat flux is not negligible but **quite weak** effect
- COMPASS data in **good correspondence with the HD model**
- **Recessed roof limiter experiment will be performed** to investigate influence of the non-continuous limiter

Thank You for your attention!



Changes in heat flux due to non-ambipolarity affect both electrons and ion (from Rob Goldston's cookbook)

$$q_{e,n-a} = q_{\parallel,e}(V=0) - q_{\parallel,e}(V=V_{float}) = 2T_e j_{sat,i} (\exp(-V_{fl}/T_e) - 1)$$

$$q_{i,n-a} = j_{sat,i} V_{fl}$$

$$q_{n-a} = q_{i,n-a} + q_{e,n-a} = 2T_e j_{sat,i} (\exp(-V_{fl}/T_e) - 1 + V_{fl}/2T_e)$$

We may relate this increase of heat flux to simple estimate $q = 7.34 * T_e * j_{sat}$

$$\frac{q_{n-a}}{q_{amb}} = \frac{2}{7.34} \left(\exp(-x) - 1 + \frac{x}{2} \right); x = V_{fl}/T_e$$

The advantage of this approach is that we don't need to know j_{sat} , which may be difficult to obtain from IVs due to problems with saturation of the ion branch

Power flux ratios (Goldston proc.)

